

**DICE: Database for the International
Handbook of Evaluated Criticality
Safety Benchmark Experiments**

Users Manual

September 2005

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What is DICE?

The “International Handbook of Evaluated Criticality Safety Benchmark Experiments” is a collection of evaluated experimental data representative of configurations encountered in the nuclear fuel cycle. The data is useful for the validation of neutronic codes and associated nuclear data libraries used for criticality-safety analysis.

The handbook is produced in electronic format (PDF files) where the experiments are grouped into evaluations, which are categorised by fissile media (Pu, Low Enriched Uranium...), fuel form (Solution, Metal...) and by neutron spectrum description (Thermal, Fast...). The evaluations are structured into different sections where the experimental program is described, the data are evaluated, the effect of experimental data uncertainties is assessed and benchmark models are discussed. Sample calculations are also presented and calculated spectral characteristics of the experiments are given.

DICE is a tool intended to make more efficient use of the handbook. Two objectives were assigned to DICE:

- Provide a summary description of each experimental configuration where the main characteristics of the experiments are displayed in a concise uniform format.
- Allow users to search the handbook for experimental configurations that satisfy their desired input criteria.

To meet these objectives, a relational database was created. Selected information from each evaluation in the handbook was entered into the database in order to provide the data required for the summaries or specific searches. A user interface was then developed to query the database and to generate different output formats (tables, summary pages, plots).

Since 2005, three different sets of evaluated experimental data can be displayed with the help of DICE: Criticality, Alarms and Physics Measurements. The feature for Physics Measurements is in place in DICE, however, first data will become available in the handbook as of the next ICSBEP version.

Requirements and compatibility

The minimum requirements for running DICE are:

- A Java Virtual Machine installed on the computer.
- A PDF reader available on the computer, e.g. Acrobat Reader.
- An HTML viewer available on the computer, e.g. Internet Explorer or Netscape.

Note that these requirements do not depend on the operating system on which the program will be run. DICE can be entirely run from the DVD, so there is no special requirement on the disk space. However, the more virtual memory that is available, the faster the execution time.

The current version of DICE offers only a Java, platform independent database that does not require any further drivers. The OracleLite database is no longer maintained.

The master database available on the NEA server can also be accessed. For that, a web connection is required (see the section on local and remote databases).

Installation

To run the users' interface, you need a Java Virtual Machine installed on your computer (Version 1.2 or higher). If Java is already installed (e.g. you already had a previous version of DICE running on your computer), please skip this step. Otherwise, the DVD provides two executables (directory dice/java) for Windows and Linux:

- double-click on the file "dice/java/j2re-1_4_2_08-windows-i586-p.exe" to install the Java runtime on Windows
- execute the file "dice/java/j2re-1_4_2_08-linux-i586-rpm.bin" to install the Java runtime on Linux.

Java runtimes for other operating systems can be obtained on the vendor's website or on the Java website (<http://www.java.com> or <http://java.sun.com>).

The installation of Java constitutes the minimum requirement for running DICE. Once Java is installed you can run the interface from the DVD and access the Mckoi Java database available on the DVD as well.

Although not necessary, DICE would run faster if the software and the database are installed on a hard drive (or a network drive). This requires about 150 MB. To do so, copy the contents of directory DICE to your drive while keeping the same sub-directories structure. Note that it is not necessary to install the entire handbook (PDF files). DICE can access these PDF files from the DVD.

Running DICE

To run DICE, either from the DVD or from a disk drive, execute the file dice.bat (for Windows) or dice.sh (for Unix).

Troubleshooting

The execution files (.bat or .sh) assume that the java command is in the users' command path. If this is not the case, the program will not run. Alternatively, you may change the execution command to specify the complete path to the java executable. For instance, if java is installed on:

C:\Program Files\Java\j2re1.4.2\bin\java, add this path between double quotation marks before the first word java. The beginning of the command will thus become:

"C:\Program Files\Java\j2re1.4.2\bin\java" -jar software\DICE.jar

DICE runs best with the Java runtime environment version 1.4 (jre-1.4) or higher. If you are using jre-1.2, you may experience some problems with the size of the windows and panels. To check the Java version installed on your computer, type the command: java -version in a DOS, UNIX... command window. If DICE is running, you may also obtain the version of Java that is running on your computer by selecting the "About D.I.C.E." option on the upper menu bar.

If the user experiences other problems with the software or with the database, please report the problem using the following procedure:

- On Windows, open a DOS command window and go to the directory where file "dice.bat" is located. Type dice.bat and copy the error messages generated by the software during the execution. Send these messages to J.Briggs@inl.gov specifying if the version of DICE used is the one originally distributed with the ICSBEP DVD or if the user has downloaded an updated version from the NEA website. In the latter case, please specify the approximate date of the last update.
- On Unix, execute the file "dice.sh" and copy the error messages generated by the software during the execution. Send these messages to J.Briggs@inl.gov specifying if the version of DICE used is the one originally distributed with the ICSBEP DVD or if the user has downloaded an updated version from the NEA website. In the later case, please specify the approximate date of the last update.

Screen resolution

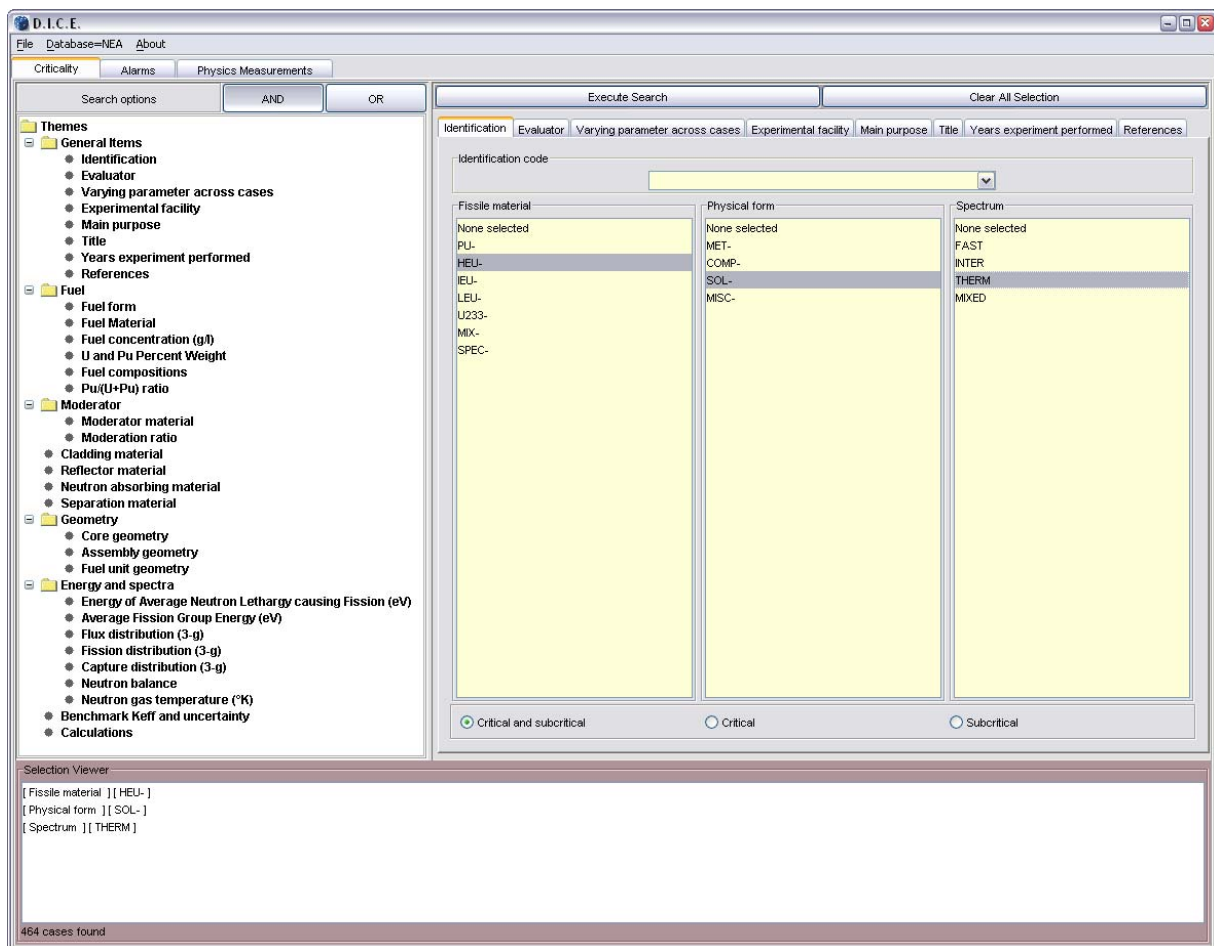
The program automatically adapts the dimensions of its windows to the screen resolution.

Using DICE

General overview

The user's interface is composed of two main screens- a "Search" screen and a "Results" screen. The selection criteria is entered using the "Search" screen and the software interrogates the database for experimental configurations satisfying the selected criteria. The paragraph on "Search themes" explains how to enter the selection criteria. The "Search" screen and "Results" screen are specific to the type of data desired: Criticality, Alarms or Physics Measurements. On the identification search page, once the identifiers are specified using the drop down menu called "Identification code", the codes satisfying your specification will be listed and one evaluation can be selected. A "Selection viewer" (panel located at the bottom of the screen) gives the list of all specified criteria. This panel is updated when a new criterion is added or deleted. To cancel a criterion, the user needs to go back to the corresponding entry in the "Search theme" and select item "None selected". To cancel all criteria, press the button "Clear All Selection".

Search screen



Results screen

The results page has two panels. The left-hand panel gives the list of evaluations in which an experimental configuration was selected. A column "# cases (NNN)" gives the number of cases in the evaluation that satisfy the criteria. NNN is the total number of cases (sum of numbers in this column). The right-hand panel gives, for each evaluation highlighted in the left-hand panel, the list of cases that satisfy the selection criteria. The complete documentation of the evaluation can be consulted by pressing the button "Display Evaluation (PDF document)". Pressing the button -Display Case- can also generate a summary of each experimental configuration, which gives a concise description of the experiment characteristics (geometry, fuel composition, moderation and reflection conditions, spectra data...).

Evaluation Identification	# cases (493)	Title
PU-SOL-THERM-001	6	WATER-REFLECTED 11.5-INCH DIAMETER SPHERE
PU-SOL-THERM-002	7	WATER-REFLECTED 12-INCH DIAMETER SPHERES
PU-SOL-THERM-003	8	WATER-REFLECTED 13-INCH DIAMETER SPHERES
PU-SOL-THERM-004	13	WATER-REFLECTED 14-INCH DIAMETER SPHERES
PU-SOL-THERM-005	9	WATER-REFLECTED 14-INCH DIAMETER SPHERES
PU-SOL-THERM-006	3	WATER-REFLECTED 15-INCH DIAMETER SPHERES
PU-SOL-THERM-007	8	WATER-REFLECTED 11.5-INCH DIAMETER SPHERE
PU-SOL-THERM-008	15	CONCRETE-REFLECTED 14-INCH DIAMETER SPHERE
PU-SOL-THERM-009	3	UNREFLECTED 48-INCH DIAMETER SPHERE OF PL
PU-SOL-THERM-010	14	WATER-REFLECTED 9-, 10-, 11-, AND 12-INCH DIA
PU-SOL-THERM-011	12	BARE 16- AND 18-INCH DIAMETER SPHERES OF P
PU-SOL-THERM-012	23	CRITICALITY OF PLUTONIUM NITRATE SOLUTION I
PU-SOL-THERM-013	21	INTERACTING CYLINDERS OF 256-mm DIAMETER V
PU-SOL-THERM-014	35	INTERACTING CYLINDERS OF 300-mm DIAMETER V
PU-SOL-THERM-015	17	INTERACTING CYLINDERS OF 300-mm DIAMETER V
PU-SOL-THERM-016	11	INTERACTING CYLINDERS OF 300-mm AND 256-mm
PU-SOL-THERM-017	18	INTERACTING CYLINDERS OF 256-mm AND 300-mm
PU-SOL-THERM-020	8	WATER-REFLECTED AND WATER-CADMIUM-REFL
PU-SOL-THERM-021	6	WATER-REFLECTED AND BARE 15.2-INCH DIAMET
PU-SOL-THERM-022	17	PLUTONIUM (19% 240Pu) NITRATE SOLUTION IN A
PU-SOL-THERM-023	34	PLUTONIUM (33.89% and 4.23% 240 Pu) NITRATE
PU-SOL-THERM-024	23	SLABS OF PLUTONIUM NITRATE SOLUTIONS REF
PU-SOL-THERM-025	44	WATER-REFLECTED SLABS OF PLUTONIUM NITR
PU-SOL-THERM-026	21	UNREFLECTED SLABS OF PLUTONIUM NITRATE S
PU-SOL-THERM-027	5	MIXED OXIDE RAPSDIE FUEL PIN ARRAYS. MODE
PU-SOL-THERM-028	15	Water Reflected Annular Cylinders (50/30 cm DIA.)
PU-SOL-THERM-029	17	INTERACTING (WATER REFLECTED OR NOT) ANNUL
PU-SOL-THERM-032	17	WATER-REFLECTED ANNULAR CYLINDERS (50/20
PU-SOL-THERM-033	63	PLUTONIUM (3.13 and 4.23% 240Pu) NITRATE SOL

Case Identification	Case Label	Core Type
PU-SOL-THERM-025-001	Experiment no. 357	Single Homogeneous Unit
PU-SOL-THERM-025-002	Exp 356, 358-9, 361	Single Homogeneous Unit
PU-SOL-THERM-025-003	Exps 355, 360	Single Homogeneous Unit
PU-SOL-THERM-025-004	Experiment no. 354	Single Homogeneous Unit
PU-SOL-THERM-025-005	Experiment no. 353	Single Homogeneous Unit
PU-SOL-THERM-025-006	Experiment no. 352	Single Homogeneous Unit
PU-SOL-THERM-025-007	Experiment no. 510	Single Homogeneous Unit
PU-SOL-THERM-025-008	Experiment no. 509	Single Homogeneous Unit
PU-SOL-THERM-025-009	Experiment no. 508	Single Homogeneous Unit
PU-SOL-THERM-025-010	Experiment no. 507	Single Homogeneous Unit
PU-SOL-THERM-025-011	Experiment no. 504	Single Homogeneous Unit
PU-SOL-THERM-025-012	Experiment no. 505	Single Homogeneous Unit
PU-SOL-THERM-025-013	Experiment no. 506	Single Homogeneous Unit
PU-SOL-THERM-025-014	Experiment no. 464	Single Homogeneous Unit
PU-SOL-THERM-025-015	Experiment no. 463	Single Homogeneous Unit
PU-SOL-THERM-025-016	Experiment no. 465	Single Homogeneous Unit
PU-SOL-THERM-025-017	Experiment no. 466	Single Homogeneous Unit
PU-SOL-THERM-025-018	Experiment no. 467	Single Homogeneous Unit
PU-SOL-THERM-025-019	Experiment no. 468	Single Homogeneous Unit
PU-SOL-THERM-025-020	Experiment no. 462	Single Homogeneous Unit
PU-SOL-THERM-025-021	Experiment no. 420	Single Homogeneous Unit
PU-SOL-THERM-025-022	Experiment no. 419	Single Homogeneous Unit
PU-SOL-THERM-025-023	Experiment no. 418	Single Homogeneous Unit
PU-SOL-THERM-025-024	Experiment no. 417	Single Homogeneous Unit
PU-SOL-THERM-025-025	Experiment no. 416	Single Homogeneous Unit
PU-SOL-THERM-025-026	Experiment no. 415	Single Homogeneous Unit
PU-SOL-THERM-025-027	Experiment no. 421	Single Homogeneous Unit
PU-SOL-THERM-025-028		Single Homogeneous Unit
PU-SOL-THERM-025-029		Single Homogeneous Unit
PU-SOL-THERM-025-030		Single Homogeneous Unit
PU-SOL-THERM-025-031		Single Homogeneous Unit
PU-SOL-THERM-025-032		Single Homogeneous Unit
PU-SOL-THERM-025-033		Single Homogeneous Unit
PU-SOL-THERM-025-034	Experiment no. 414	Single Homogeneous Unit
PU-SOL-THERM-025-035	Experiment no. 413	Single Homogeneous Unit
PU-SOL-THERM-025-036	Experiment no. 412	Single Homogeneous Unit
PU-SOL-THERM-025-037	Experiment no. 411	Single Homogeneous Unit
PU-SOL-THERM-025-038	Experiment no. 410	Single Homogeneous Unit
PU-SOL-THERM-025-039	Experiment no. 461	Single Homogeneous Unit
PU-SOL-THERM-025-040	Experiment no. 460	Single Homogeneous Unit
PU-SOL-THERM-025-041	Experiment no. 459	Single Homogeneous Unit
PU-SOL-THERM-025-042	Experiment no. 458	Single Homogeneous Unit
PU-SOL-THERM-025-043	Experiment no. 457	Single Homogeneous Unit
PU-SOL-THERM-025-044	Experiment no. 456	Single Homogeneous Unit

In addition to the summary pages, the user may want to access specific parameters for all the selected experiments (e.g. variation range of fuel concentration for solutions, moderation to fuel volume ratio in lattices...). This may be useful for parametric analysis of calculation results or for studying correlations between physical parameters. In fact, the "Result" screen contains two tabs located just above the "Display Evaluation (PDF document)" button:

- "Results - Summary of evaluations and cases found" lists evaluations and cases meeting the search criteria (previously discussed).
- "Results - Selection of details to display" enables the selection of parameters to be displayed in a tabular format (see next screen copy).

When the latter tab is selected, a list of items is displayed from which the user may choose the ones he wants to generate in a tabular format. After selecting the desired parameters and pressing the “View data button”, a table is constructed in the lower part of the page. The user may change the order of columns by a simple mouse drag and drop. Also, the table may be sorted by ascending order of any of the parameters displayed in the column. The default sorting is ascending order of the first column, which is the experiment identification. To change the sorting order, click on the title of a column and the whole table will be sorted according to the parameter contained in this column. The current version of DICE does not allow plotting graphs of these data. However, the table may be exported into a file using the “Export data to file” button. A dialog window will ask the user to choose the character to be used for separating the values (semicolon, blank...) and for the location and name of the file. The generated tabular file may be used in any graphical package (e.g. MS Excel) to plot graphs.

The screenshot shows the D.I.C.E. software interface. The 'Results' tab is active, displaying a table of experimental data. The table has columns for Identification, Reflector material, Benchmark Keff, Benchmark Keff uncertainty (1 sigma), Code Name, Library, Keff, Keff Uncertainty, and Keff / Benchmark Keff. The data rows show various experimental configurations, including different fuel materials (Light Water, Water + Cadmium) and moderators (KENO, MCNP).

Identification	Reflector material	Benchmark Keff	Benchmark Keff uncertainty (1 sigma)	Code Name	Library	Keff	Keff Uncertainty	Keff / Benchmark Keff
PU-SOL-THERM-020-001	Light Water	1	0.0059	KENO	ENDF/B-IV 27-Group	1.0151	0.0025	1.0151
PU-SOL-THERM-020-001	Light Water	1	0.0059	KENO	Hansen-Roach	1.0007	0.0028	1.0007
PU-SOL-THERM-020-001	Light Water	1	0.0059	MCNP	ENDF/B-V Continuous	1.0094	0.001	1.0094
PU-SOL-THERM-020-002	Light Water	1	0.0059	KENO	ENDF/B-IV 27-Group	1.0183	0.0026	1.0183
PU-SOL-THERM-020-002	Light Water	1	0.0059	KENO	Hansen-Roach	1.0042	0.0025	1.0042
PU-SOL-THERM-020-002	Light Water	1	0.0059	MCNP	ENDF/B-V Continuous	1.0123	0.0009	1.0123
PU-SOL-THERM-020-003	Light Water	1	0.0059	KENO	ENDF/B-IV 27-Group	1.0092	0.0025	1.0092
PU-SOL-THERM-020-003	Light Water	1	0.0059	KENO	Hansen-Roach	0.9993	0.0025	0.9993
PU-SOL-THERM-020-003	Light Water	1	0.0059	MCNP	ENDF/B-V Continuous	1.0088	0.001	1.0088
PU-SOL-THERM-020-004	Light Water	1	0.0059	KENO	ENDF/B-IV 27-Group	1.0212	0.0024	1.0212
PU-SOL-THERM-020-004	Light Water	1	0.0059	KENO	Hansen-Roach	1.0033	0.0028	1.0033
PU-SOL-THERM-020-004	Light Water	1	0.0059	MCNP	ENDF/B-V Continuous	1.0122	0.0009	1.0122
PU-SOL-THERM-020-005	Light Water	1	0.0059	KENO	ENDF/B-IV 27-Group	1.0157	0.0025	1.0157
PU-SOL-THERM-020-005	Light Water	1	0.0059	KENO	Hansen-Roach	1.0032	0.0028	1.0032
PU-SOL-THERM-020-005	Light Water	1	0.0059	MCNP	ENDF/B-V Continuous	1.0126	0.001	1.0126
PU-SOL-THERM-020-006	Light Water	1	0.0059	KENO	ENDF/B-IV 27-Group	1.0053	0.0024	1.0053
PU-SOL-THERM-020-006	Light Water	1	0.0059	KENO	Hansen-Roach	0.9927	0.0027	0.9927
PU-SOL-THERM-020-006	Light Water	1	0.0059	MCNP	ENDF/B-V Continuous	1.0071	0.0009	1.0071
PU-SOL-THERM-020-007	Water + Cadmium	1	0.0059	KENO	ENDF/B-IV 27-Group	1.0159	0.0029	1.0159
PU-SOL-THERM-020-007	Water + Cadmium	1	0.0059	KENO	Hansen-Roach	0.996	0.0029	0.996
PU-SOL-THERM-020-007	Water + Cadmium	1	0.0059	MCNP	ENDF/B-V Continuous	1.0119	0.001	1.0119
PU-SOL-THERM-020-008	Water + Cadmium	1	0.0059	KENO	ENDF/B-IV 27-Group	0.9983	0.0025	0.9983
PU-SOL-THERM-020-008	Water + Cadmium	1	0.0059	KENO	Hansen-Roach	0.991	0.0032	0.991
PU-SOL-THERM-020-008	Water + Cadmium	1	0.0059	MCNP	ENDF/B-V Continuous	1.0023	0.001	1.0023

Summary table

This table is generated when the button "Display case" is pressed (see above). It summarizes each experimental configuration using the parameters stored in the database. The summary table depends on the set of data searched (Criticality, Alarms).

Criticality:

Some of the entries are common to all the configurations contained within the same evaluation (evaluators, publication, and revision dates, title, main purpose of the experimental program, varying parameters across cases, fuel material, name of the laboratory where the experiments were performed, dates the experiments were performed, and the references). Other types of information are specific to one case and concern the description of the geometry, composition, moderation and reflection characteristics, spectra data, benchmark Keff and associated uncertainty, and sample calculations.

Additional details are given here for some of these categories. The table is given in HTML format and can be saved to disk (right click the mouse while selecting the row corresponding to a specific case).

Geometry description:

To describe the geometry of the experiment, three levels of details were designated.

1) *Core geometry*: A description of the overall configuration is given. The possible choices are: Single Homogeneous Unit, Single Assembly, Array of Clad or Unclad Homogeneous Units, Array of Assemblies, or Complex Geometry.

2) *Assembly description*: In the case of configurations containing assemblies, a description is given of the assembly. The possible choices are: Lattice of Fuel Pins or Tubes, Fuel Plates, or Complex Geometry.

3) *Basic fuel unit*: Finally the elementary fuel unit is described (pincell in the case of an assembly or the entire unit for homogeneous configurations). The list of choices is much larger in this case.

In the case of complicated shapes for the core or for the assembly, a summary description is given as a free text.

Composition

The database contains data for fuel compositions. When a configuration contains multiple fissile units of different compositions, all compositions were entered when judged useful (see the discussion below on configurations containing different fissile units). However, some experiments contain several fissile media that differ only slightly. Only a typical or average value is provided on the database. For example, multiple fuel compositions are used in each configuration given in HEU-COMP-INTER-003, but only a typical value is entered into the database.

The data entered into the database are atom densities in atoms/(barn-cm) as given in section 3.3 of the evaluation. However, these data are not displayed in the summary. The data are used, rather, to compute the uranium and plutonium isotopic vectors in weight per cents. The atom densities are also used to compute the moderator-to-fuel and the moderator-to-fissile density ratios for homogeneous fuels.

Moderator-to-fuel ratio

Depending on the fuel configuration the moderator-to-fuel ratio is calculated as the ratio of atomic densities (homogeneous media) or the ratio of respective volumes (assembly type geometry). For homogeneous media, the moderator-to-fissile atomic densities ratio is also given. For simple cases, this information is automatically calculated from the geometry and composition data. For complex situations, this information was calculated by hand and entered into the database with the mention of the moderation ratio type (atom density, volume ratio...).

Configurations containing different fissile units

Some experiments contain more than a single type of fissile unit. The difference can be the assembly description (a typical example is reactor experiments with a driver assembly and a test assembly of different pitches), fuel form or composition. For such configurations, the fuel descriptions are entered for the different units and are displayed in the summary.

Alarms:

The entries common to all configurations are: evaluators, publication, and revision dates, title, main purpose of the experimental program, name of the laboratory where the experiments were performed, dates the experiments were performed, additional comment, and the references. Other types of information are specific to one case and concern the description of the source and detector.

Source

One or various sources can be used in the same experiment and entered into the database. The type of source, the particle, time dependency, neutron strength and/or gamma strength and the geometry of the source are summarised in the table.

Detector

The different detectors used during the experiment and the analysis are specified using the type of detector, size and efficiency.

Plotting capabilities

Plotting capabilities were implemented into DICE to allow the user to view graphical representations of neutron spectra such as the flux, reaction rates or sensitivity coefficients. The information used by DICE for the plotting is contained in external text files, i.e. it is not contained within the database.

Once a search is performed, the following buttons appear below the menu bar:

New Spectra plot

New Sensitivity plot

Close current plot

Close all plots

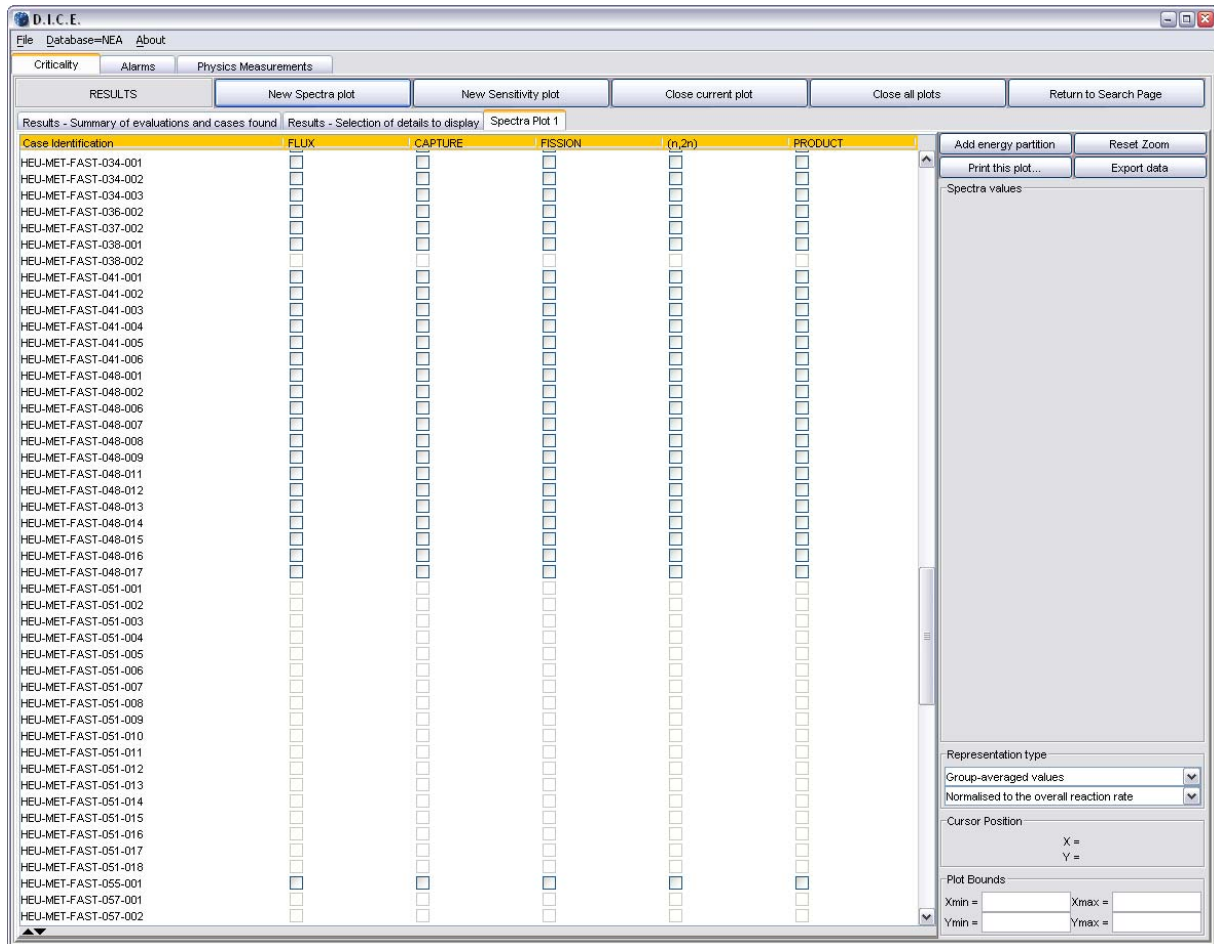
Return to Search Page

The screenshot shows the D.I.C.E. (Database Interface for Criticality Evaluation) software interface. The main window displays a search results table with the following columns: Evaluation Identification, # cases (248), Title, Case Identification, Case Label, and Core Type. The table lists various nuclear reactor configurations and their associated case labels and core types. The interface includes a menu bar (File, Database=NEA, About) and a toolbar with buttons for 'New Spectra plot', 'New Sensitivity plot', 'Close current plot', 'Close all plots', and 'Return to Search Page'. The table is divided into two main sections: 'Display Evaluation (PDF document)' and 'Display Case'. The 'Display Case' section is currently selected, showing a list of cases with their labels and core types.

Evaluation Identification	# cases (248)	Title	Case Identification	Case Label	Core Type
HEU-MET-FAST-012	1	SPHERE OF HIGHLY ENRICHED URANIUM REF	HEU-MET-FAST-071-001	1	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-013	1	SPHERE OF HIGHLY ENRICHED URANIUM REF	HEU-MET-FAST-071-002	2	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-014	1	SPHERE OF HIGHLY ENRICHED URANIUM REF	HEU-MET-FAST-071-003	3	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-015	1	UNREFLECTED CYLINDER OF HIGHLY ENRICH	HEU-MET-FAST-071-004	4	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-016	2	BERYLLIUM-REFLECTED AND BERYLLIUM OX	HEU-MET-FAST-071-005	4a	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-017	1	BERYLLIUM-MODERATED AND -REFLECTED	HEU-MET-FAST-071-006	5	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-018	1	BARE SPHERICAL ASSEMBLY OF 235 U(90%)	HEU-MET-FAST-071-007	5a	Array of Assemblies
HEU-MET-FAST-019	1	GRAPHITE-REFLECTED SPHERICAL ASSEMBL	HEU-MET-FAST-071-008	6	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-020	1	POLYETHYLENE-REFLECTED SPHERICAL ASS	HEU-MET-FAST-071-009	7	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-021	1	STEEL-REFLECTED SPHERICAL ASSEMBLY O	HEU-MET-FAST-071-010	8	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-022	1	DURALUMIN-REFLECTED SPHERICAL ASSEMB	HEU-MET-FAST-071-011	9	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-023	22	TINKERTOY: UNMODERATED URANIUM METAL	HEU-MET-FAST-071-012	10	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-024	1	SPHERE OF HIGHLY ENRICHED URANIUM REF	HEU-MET-FAST-071-013	11	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-026	32	TINKERTOY 2: BARE AND PARAFFIN-REFLEC	HEU-MET-FAST-071-014	12	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-027	1	SPHERICAL ASSEMBLY OF 235 U(90%) WITH	HEU-MET-FAST-071-015	12a	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-028	1	URANIUM-235 SPHERE REFLECTED BY NORM	HEU-MET-FAST-071-016	12b	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-029	1	SPHERICAL ASSEMBLY OF 235 U(90%) WITH	HEU-MET-FAST-071-017	13	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-030	1	HETEROGENEOUS CYLINDER OF HIGHLY ENR	HEU-MET-FAST-071-018	14	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-031	1	SPHERICAL ASSEMBLY OF 235 U(90%) WITH	HEU-MET-FAST-071-019	15	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-032	4	235 U(94%) SPHERES SURROUNDED BY NA	HEU-MET-FAST-071-020	16	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-034	3	THREE HETEROGENEOUS CYLINDERS OF HIG	HEU-MET-FAST-071-021	16a	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-036	1	TWO HETEROGENEOUS CYLINDERS OF HIGH	HEU-MET-FAST-071-022	17	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-037	1	TWO HETEROGENEOUS CYLINDERS OF HIGH	HEU-MET-FAST-071-023	18	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-038	2	TWO HETEROGENEOUS CYLINDERS OF HIGH	HEU-MET-FAST-071-024	19	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-041	6	235 U(94%) SPHERES SURROUNDED BY BEF	HEU-MET-FAST-071-025	20	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-048	13	OIL REFLECTED SPHERES AND HEMISPHERES	HEU-MET-FAST-071-026	21	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-051	18	SINGLE SOLID URANIUM (93.2) METAL CYLIN	HEU-MET-FAST-071-027	22	Array of Clad or Unclad Homogeneous Units
HEU-MET-FAST-055	1	ZPR-3 ASSEMBLY 23: A CYLINDRICAL ASSE			
HEU-MET-FAST-057	6	CRITICAL ASSEMBLIES OF HIGHLY ENRICHED			
HEU-MET-FAST-058	5	HIGHLY ENRICHED URANIUM METAL SPHERES			
HEU-MET-FAST-060	1	ZPR-9 ASSEMBLY 4: A CYLINDRICAL ASSEMB			
HEU-MET-FAST-061	1	ZPR-21 PHASE F: A CYLINDRICAL ASSEMB			
HEU-MET-FAST-062	1	FIRST CRITICALITY OF CORAL-I REACTOR, A			
HEU-MET-FAST-063	2	CRITICAL EXPERIMENTS PERFORMED USING H			
HEU-MET-FAST-064	3	THREE CYLINDERS OF LEAD-REFLECTED HIG			
HEU-MET-FAST-065	1	UNREFLECTED CYLINDER OF HIGHLY ENRICH			
HEU-MET-FAST-066	9	METAL SPHERICAL SHELLS OF HIGHLY ENRICH			
HEU-MET-FAST-067	2	ZPR-9 ASSEMBLIES 5 AND 6: HEU (93% 235U)			
HEU-MET-FAST-068	1	CRITICAL ASSEMBLIES COMPOSED OF HIGHLY			
HEU-MET-FAST-070	3	ZPR-9 ASSEMBLIES 7, 8 AND 9: CYLINDRICAL			
HEU-MET-FAST-071	27	URANIUM (93.2) METAL ANNULI WITH ONE AN			
HEU-MET-FAST-072	2	ZELUS: FAST-SPECTRUM CRITICAL ASSEMBLIES			
HEU-MET-FAST-073	1	THE UNMODERATED ZELUS EXPERIMENT: A C			
HEU-MET-FAST-075	1	ZPR-20 PHASE C: A CYLINDRICAL ASSEMBLY			
SUB-HEU-MET-FAST-001	1	ZPR-20 PHASE E: A CYLINDRICAL ASSEMBLY			

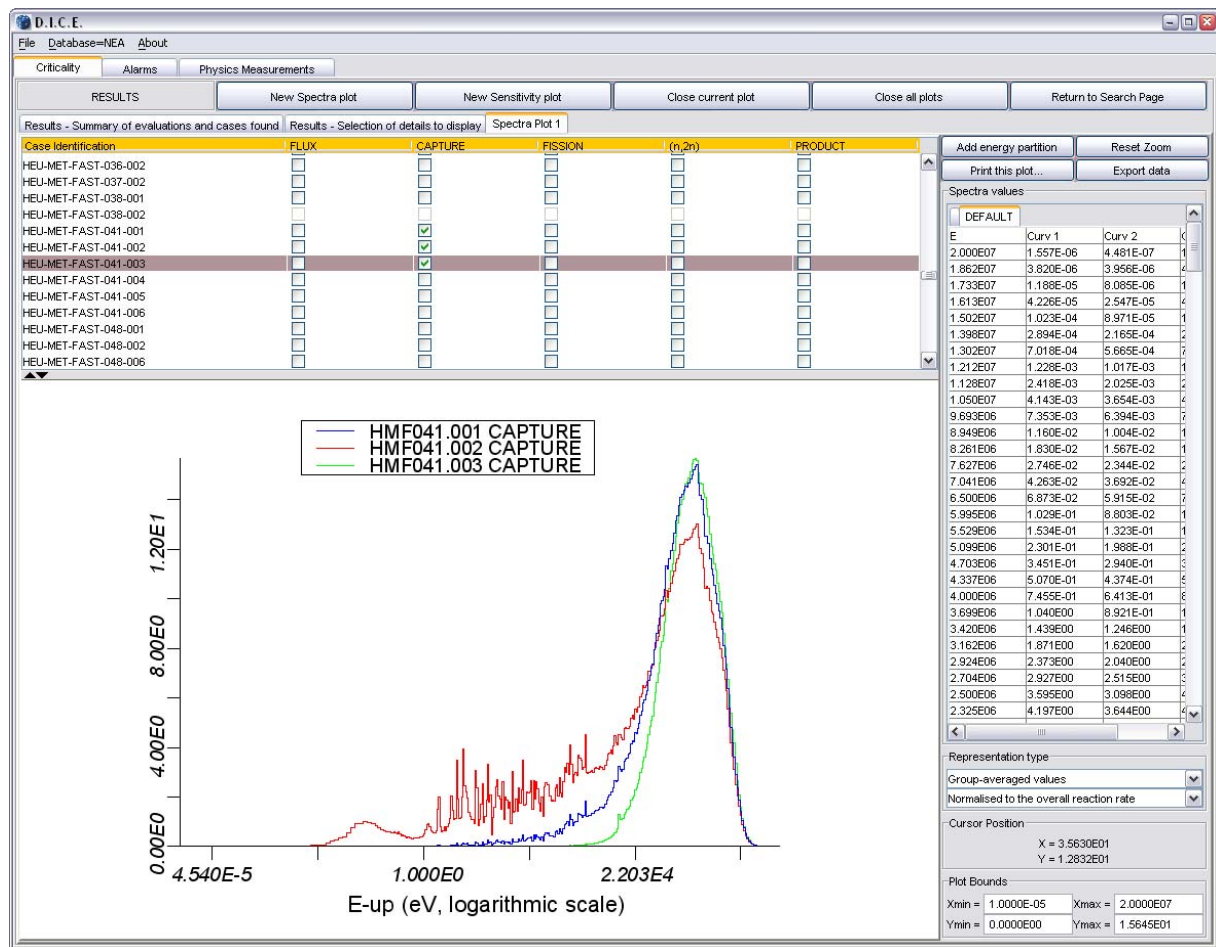
How to plot neutron spectra

When pressing the button “New Spectra plot”, a new tab is created and is designated as “Spectra Plot 1” (or another number when the “New Spectra plot” button is pressed more than once). The following figure gives a screen capture to illustrate the layout. The panel that appears below the tabs is a table with the identification number of the configurations in the first column followed by columns for the flux, capture, fission, (n,2n) and production reaction rates. In each cell of the table, a selection mark is provided.



Checking one of these marks would generate a plot of the corresponding spectrum (given by the selected column) for the selected configuration (the highlighted line). This is reflected in the legend of each graph (see the figure below). Several selection marks can be checked thus enabling the user to compare the spectra data of various configurations. The gray marks (non-selectable marks) means that the spectra data are not available for this configuration. This can be due to the fact that the spectra data for this configuration were not supplied with this version of DICE. In fact, about 10% of the configurations in the 2004 edition do not have corresponding spectra files. However, if all marks are gray, this means that the software is not looking in the right directory for spectra files. In such a case, refer to the “DICE settings” to correct the path to the spectra files.

DICE allows the user to plot all the fluxes and reaction rates in a selection panel without individually selecting all the marks. If the user is, for instance, interested in plotting all the fluxes shown in the panel, then a double click in the yellow cell “Flux” will cause the software to systematically check all the active selection marks in the “Flux” column. To deselect all, simply click on button, “Close current plot”.



The data used to generate these plots were calculated using the KENO-V.a and KENO-VI codes of the ORNL SCALE code system¹, the Russian Monte Carlo code MMK-KENO and ABBN-93 299-energy group cross-section data.

The original flux and reaction rates are given in a 299-group energy mesh (referred to as the default mesh). The energy grids are given in the right-hand side panel.

The mouse can be used to zoom to a particular region of the graph. In this case, by using the mouse, the user can simply draw a rectangle in the plotting area, which will define the [xmin, xmax] and the [ymin, ymax] boundaries. Alternatively, the user can enter specific values for these limits in the "Plot Bounds" frame located at the right-bottom part of the screen. At any time the user can come back to the default zoom level (whole energy range and maximum range for the y-axis) by right clicking with the mouse on the graph or by pressing the button "Reset Zoom" located at the top of the right panel. Other buttons are available in the same panel to print the graph and to close the current graph. On the level just above, other buttons are available to close all graphics, or to create other graphical areas that will be added up as leaflets to be accessed with corresponding tabs.

Several options were in place to allow the user to customise the graphical display.

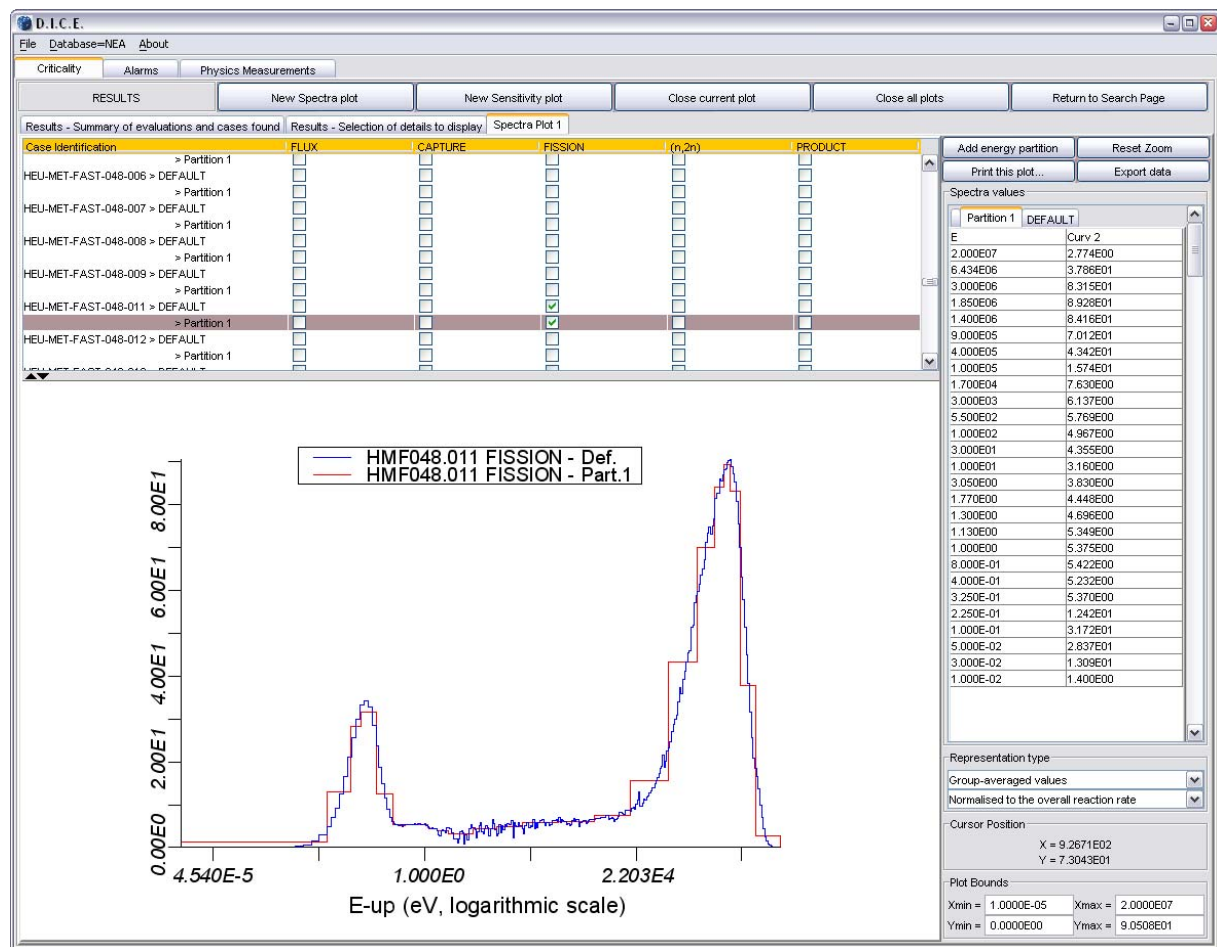
¹ RSICC CCC-545 "SCALE: A Modular Code System for Performing Standardized Computer Analyses for Licensing Evaluation", ORNL/NUREG/CSD-2/V2/R5.

Changing the energy structure

The user may want to plot the spectra on a broader energy mesh. Starting from the 299-group structure, this might be accomplished by collapsing the information to a broader structure assuming that the reaction rate within each fine group (one of the 299 groups) is constant. While this assumption is valid when each of the broad groups covers several fine groups, the user should keep in mind the underlying approximation, i.e. that the original data is not available in a structure finer than the 299 groups.

The button “Add energy partition” located at the top of the right-hand side panel allows the user to browse the computer’s disks and directories and choose the name of a text file containing the energy grids of the new energy structure (see DICE Settings options for setting the path to the desired directory). This text file should be structured so that the energies are given in descending order, one energy value per line. Examples are provided in directory data/newE.

Once a new energy structure is selected, the selection table is updated with the addition of a new line of selection buttons. For each configuration, the user now has the choice of plotting the spectra in the original energy structure (299-group) designated as “DEFAULT” and in any of the new structures designated as “partition 1”, “partition 2”. The interface offers the flexibility for plotting any combination of energy structure, reaction rate and configuration. An example is shown in the following screen capture, where the flux from the same configuration is represented in two different energy structures.



Choosing the type of representation

The calculated flux and reaction rates contained in the text files and used as primary data for the spectra plotting were defined in that the value in each group is **integrated** over the energy limits of the group. The sum over the whole energy range is equal to *1 000 x the reaction rate*, i.e. the data is **normalised** to **1 000 neutrons** emitted in the system.

As the widths of the energy groups in the 299-group structure are not constant, the default option used by DICE is a group-averaged representation (per unit lethargy) instead of a group-integrated representation. The user might change to a group-integrated representation by selecting the appropriate option in the first selection list located at the right-hand side just above the “Cursor Position” frame.

On the other hand, since the magnitude of the flux and the reaction rates varies from one configuration to the other, the comparison of spectra across configurations may not be simple if they are not normalized to the same value. DICE offers the possibility to change the normalisation option from the default option of “Normalized to the overall reaction rate” to the one in which all spectra are normalised to 1.

Note that the two aforementioned choices should be done before the plotting action is activated. Changing the options will not affect the spectra that are already plotted on the graph. In other words, if the user plots a graph with a given choice and wants to change this choice, he has to first deactivate the plot, change his choice of representation and then re-plot.

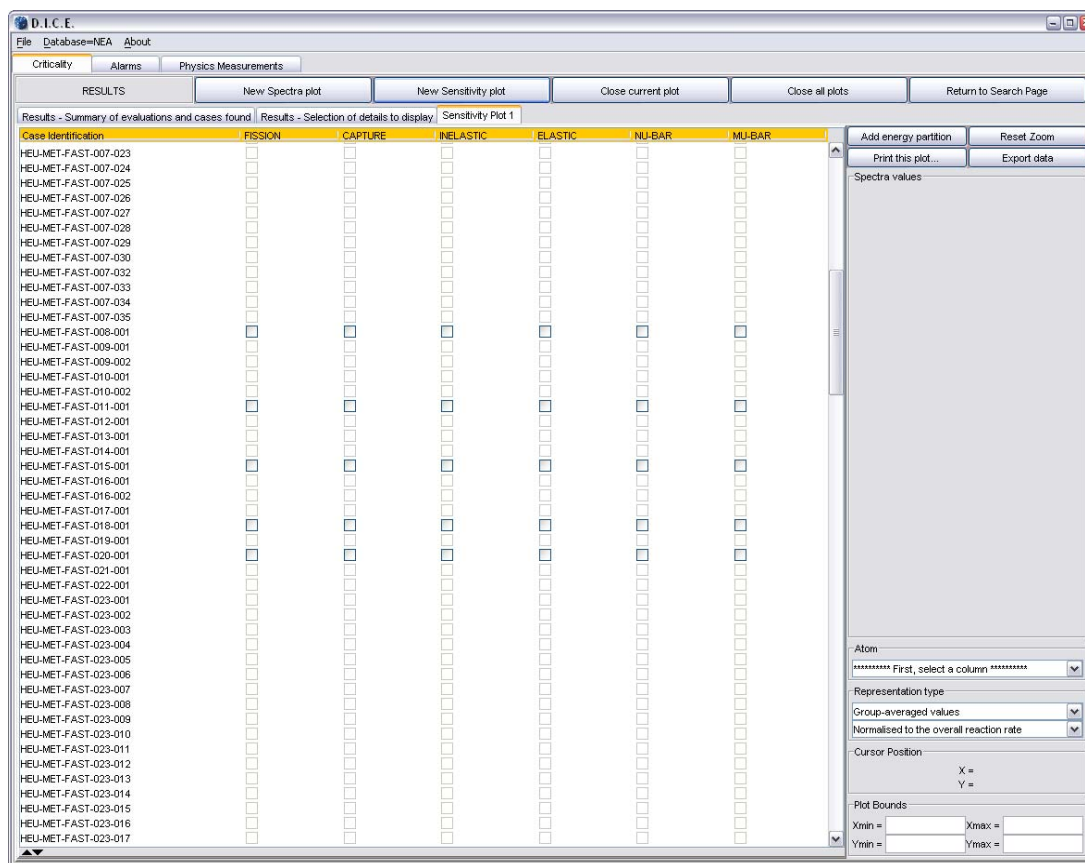
How to plot sensitivity coefficients

Sensitivity coefficients are defined as a per cent change of the K_{eff} of a given configuration subject to a one per cent change of a particular nuclear data of a certain nuclide in a given energy group. The considered nuclear data processes are: capture, fission, elastic scattering, inelastic scattering, ν -bar (average number of neutrons emitted per fission) and μ -bar (average cosine of scattered neutrons). The sensitivity coefficients were calculated using the Monte Carlo code MMK-KENO as well as the KEFSF and KEFSFSPH² codes on the basis of the transport equation solution by the TWODANT code and the ABBN-93 299-energy-group cross-section data.

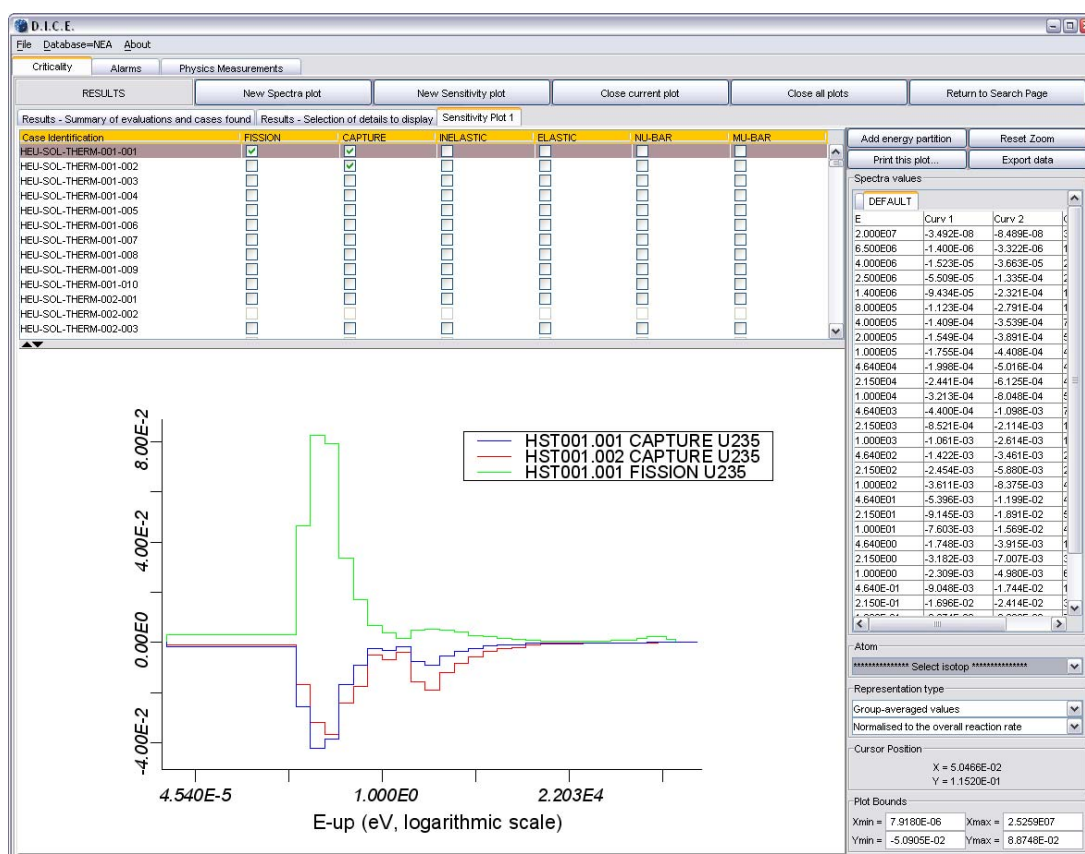
Sensitivity coefficients are currently available in a 30-group energy structure for a selected set of configurations (HEU-SOL-THERM). A directory structure (under data\sensitivity) is used to locate these files (see also the paragraph on DICE Settings). Note that the user might add additional files as they become available and access them through DICE. However, the format of the files should be identical to the format provided on the DVD.

DICE implements an interface to access these files and to plot the sensitivity profiles. A button “New sensitivity plot” is available for this purpose. Once this button is pressed, a table is displayed (see the following screen capture), which allows the selection of the configuration and the nuclear process.

² KEFSH and KEFSFSPH are tools developed by IPPE for sensitivity studies. These codes are not yet available in the public domain.



After that, the user needs to select the nuclide (“Atom”) in the scrolling list of available nuclides in the composition. The following graph shows examples of such plots.



The options available for the manipulation of the graphs are comparable to those available under the “Spectra plot” feature.

Search themes

The search criteria are grouped into several themes. The left-hand panel “Selection Tree” displays the criteria where a theme is symbolized by a folder. Double click on the folder or simple-click on the plus (+) sign to expand the folder and to see the criteria included in a theme.

When selecting a theme in the “Selection Tree”, the central panel “Selection Panel” displays different tabs, each one corresponding to a search criterion. There is a full correspondence between the “Selection Tree” and the “Selection Panel”. To move to a specific criterion within a theme, use either the corresponding tab in the “Selection Panel” or the corresponding bullet in the “Selection Tree”.

The selection criteria belong to the following categories. An example is given for each category.

Free text

- References
- Title

Selection list

- Varying parameters across cases
- Evaluator
- Evaluation identification
- Experimental facility
- Main purpose
- Isotopes present in the fuel composition
- Fuel form
- Fuel material
- Moderator material
- Cladding material
- Reflector material
- Separator material
- Fuel unit geometry (Shape)
- Source particle
- Source geometry

Range of integer values

- Years experiment performed

Range of real values

- Fuel concentration
- Pu/(U+Pu) ratio
- Spectra characteristics (EALF, AFGE, 3-group distributions of flux, fission and capture rates, neutron gas temperature, Keff and associated uncertainty)
- Detector efficiency
- Source strength

Combination of selection list and range of real values

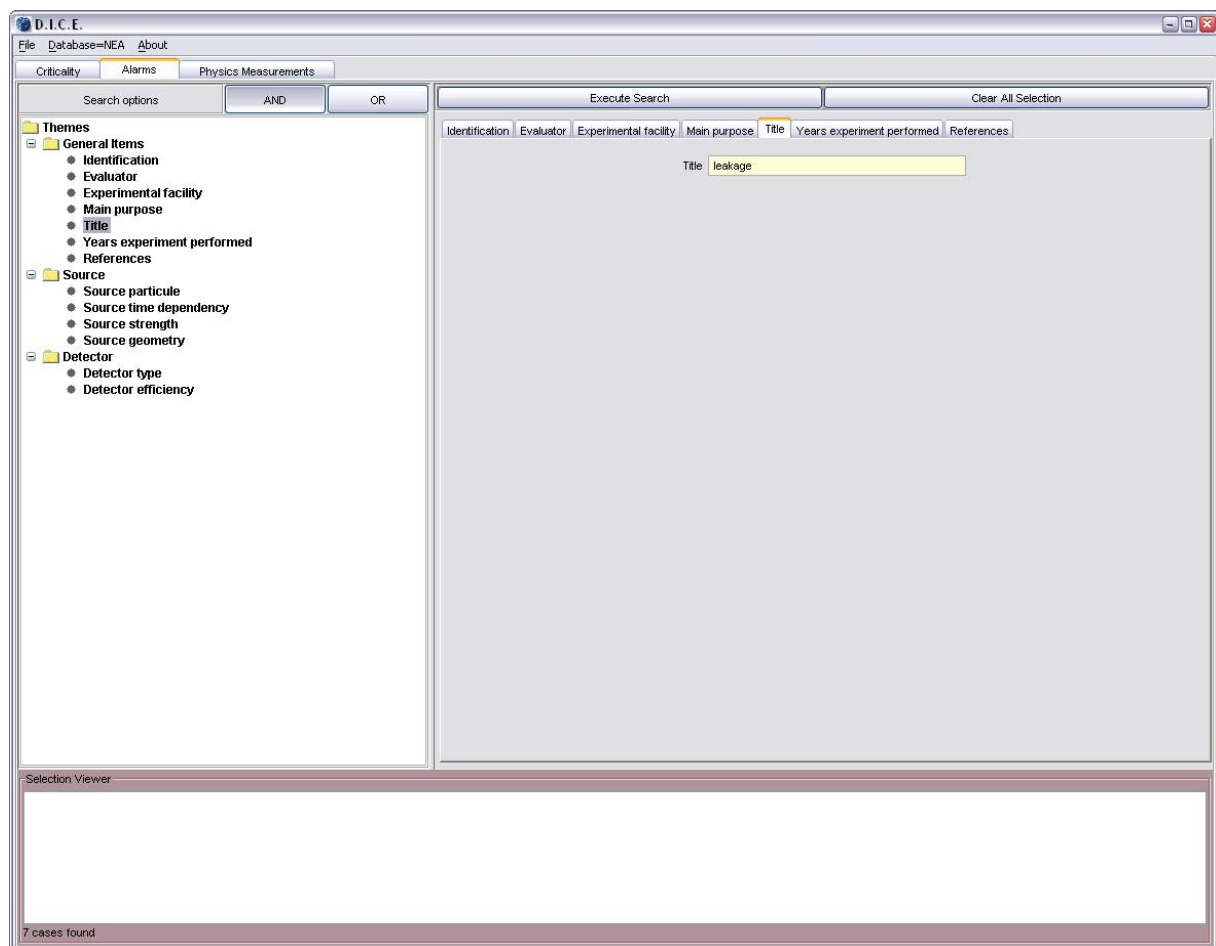
- Core geometry (shape of the core and number of fuel units)
- Moderation ratio (moderation ratio expression – atomic ratio, volume ratio... - and corresponding value)

- Neutron-absorbing material (solid or soluble form and concentration)
- Neutron balance (contribution of a particular isotope to the fission or capture in the core)
- Calculations (results obtained with a combination of code and nuclear data library)
- Assembly geometry description (shape of the assembly, pitch type and value)
- Source time dependency
- Detector type

Operators “AND” and “OR” can be used to combine *different* selection criteria. For instance, if the user wants to find experiments performed in the 1960s containing nitrate uranium fuel, the operator “AND” should be used to combine the criteria on “Years experiments performed” and “Fuel material”. Operator “AND” is used by default to combine multiple criteria. However, if the user is interested in experiments where iron is used either as a separator material or as a reflector then operator “OR” should be used to combine the corresponding criteria.

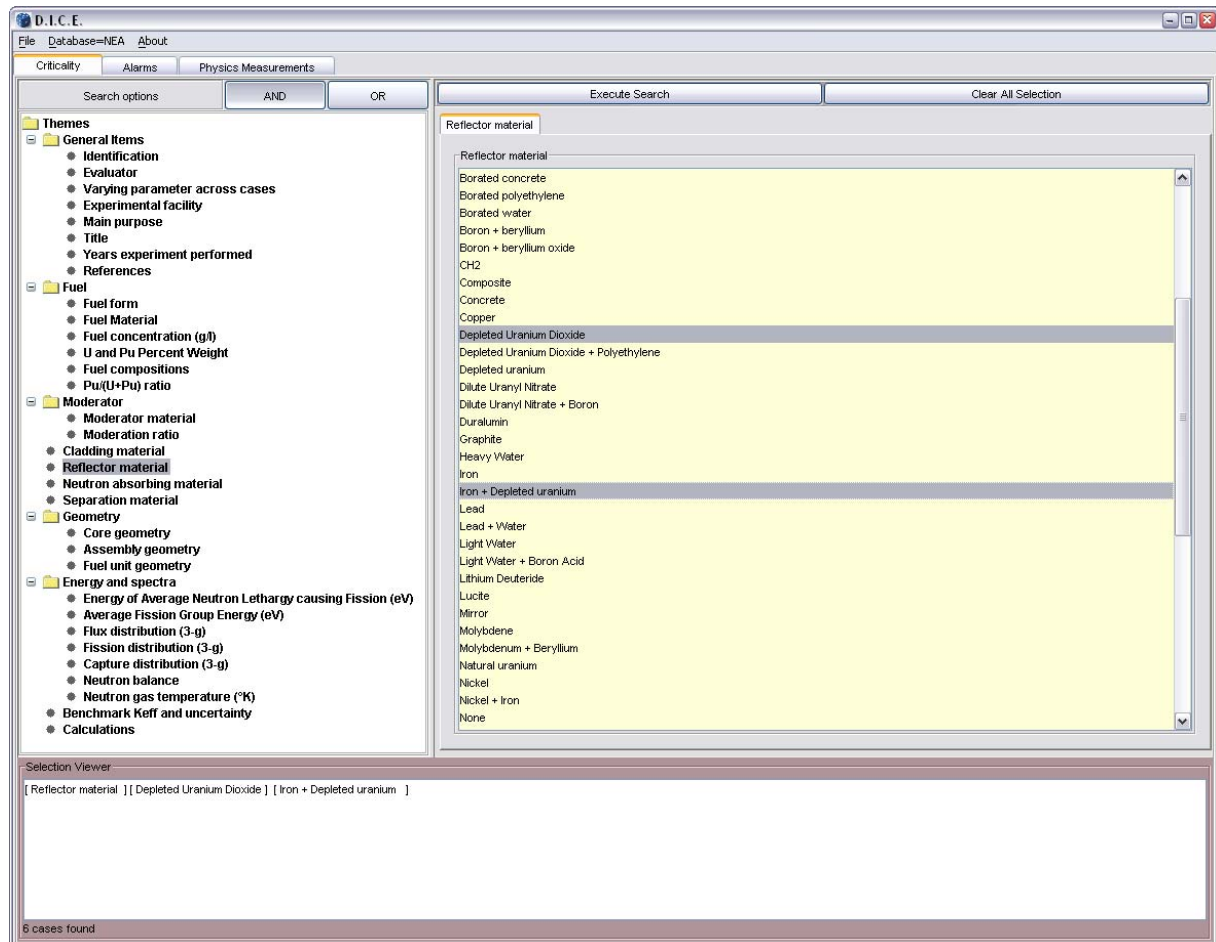
Example of a free text field selection

Enter one or multiple words separated with a comma and choose between the two operators OR/AND. The program will search the occurrence of these words according to the specified option.



Example of a selection in a selection list

In all selection lists, the first item is “None selected”. This allows cancelling a selected item in the list. Multiple selections are possible by holding down the Shift or Ctrl key. If several items are selected, and the user needs to cancel one of them, press down on the Ctrl key and re-select this item.



Example of a selection in range of real values

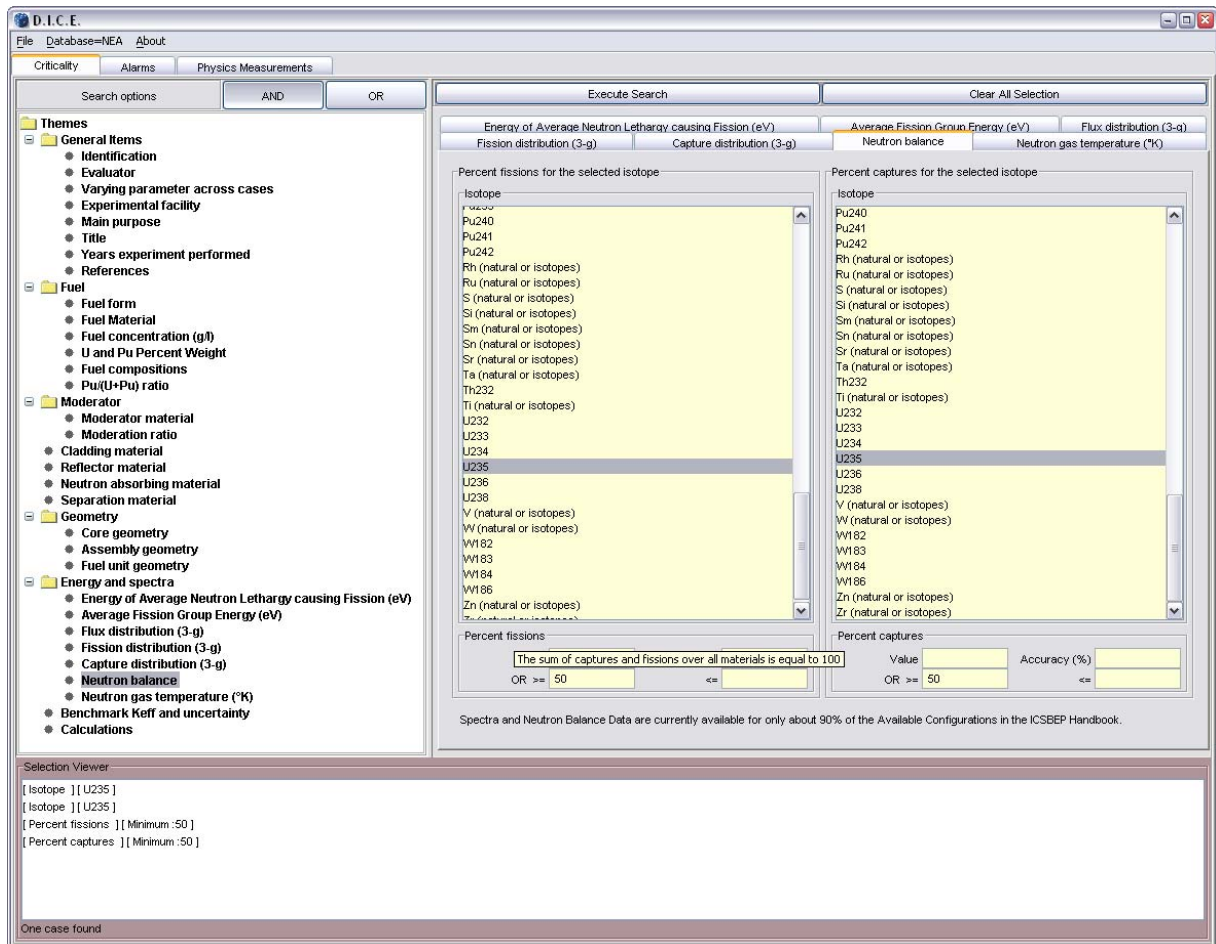
Specify the parameter value (in the specified units) with a percent accuracy, e.g. 100 with an accuracy of 10% would return values between 90 and 110. Alternatively, the range can explicitly be entered (minimum and maximum values). The inequalities are not strict.

The screenshot shows the D.I.C.E. (Database Interface for Criticality Evaluation) software window. The interface is divided into several sections:

- Top Menu:** File, Database=NEA, About.
- Search Options:** Search options, AND, OR.
- Themes:** A tree view on the left showing categories like General Items, Source, and Detector.
- Search Configuration:** A central area with tabs for Source particle, Source time dependency, Source strength, and Source geometry. The 'Source strength' tab is active, showing fields for 'Source neutrons strength (n/s)' and 'Source gamma strength (g/s)'. The 'Source neutrons strength (n/s)' field has a 'Value' of 5e+8 and an 'Accuracy (%)' of 10. The 'Source gamma strength (g/s)' field has a 'Value' of 1e+8 and an 'Accuracy (%)' of 10.
- Selection Viewer:** A bottom section showing the results of the search. It displays the text '[Source neutrons strength (n/s)] [Minimum :5e+8]'.
- Footer:** A status bar at the bottom indicating '18 cases found'.

Example of a selection in a combination of selection list and range of real values

For some search criteria, there is correspondence between the selection list and the range of entered values. This means that this range will be applied to the specific item selected in the list. In the following example, a search will be conducted for experiments in which the U-235 contribution to fission and capture were higher than 50%.



Example of a selection in a range of integer values

“Years experiments performed” is the only one in this category. For example, one may search experiments performed in the 1980s. In this case, experimental programs started before 1980 but ended after 1980 will be counted as well. One can also search for experiments performed before (or after) a certain year. In this case, the specified years (minimum and maximum) are counted.

Note: For a few experimental programs, there was a discontinuity in the operation. For instance, experiment HEU-MET-FAST-008 was performed in 1982 and in 1995. They are treated in the database as being performed over the whole period of time without discontinuity.

The screenshot shows the D.I.C.E. (Database Interface for Criticality Experiments) software. The interface is divided into several sections:

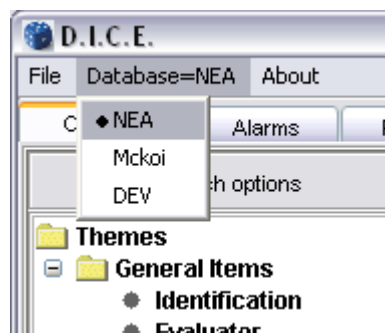
- Top Bar:** Includes a menu bar (File, Database=NEA, About) and tabs for Criticality, Alarms, and Physics Measurements.
- Search Options:** Features a tree view on the left under "Themes" with categories like General Items, Fuel, Moderator, Geometry, and Energy and spectra. The "Years experiment performed" item is selected.
- Search Criteria:** A central panel with "Execute Search" and "Clear All Selection" buttons. It shows the selected criteria: "Years experiment performed" with a range from "Begin: 1970" to "End: 1980".
- Selection Viewer:** A bottom panel showing the search results. It displays the selected criteria and the number of cases found: "1242 cases found".

Local and remote databases

The DVD provides a local database in directory Dice\databases. The local database is a Mckoi, or Java database, which should run on almost all platforms. The contents of the database reflect the status of data entries at the date of publication.

The central database is located on the NEA website where corrections and additions are implemented as needed. This database is to be utilised if the user wishes to access the latest version of the data entries. The connection is made through the Java Remote Method Invocation technique and does not require any extra driver or software on the user's computer, except for a web connection.

The menu "database" enables the user to switch from one version to another by choosing either: "Mckoi" or "NEA" option. The default database at startup is "Mckoi".



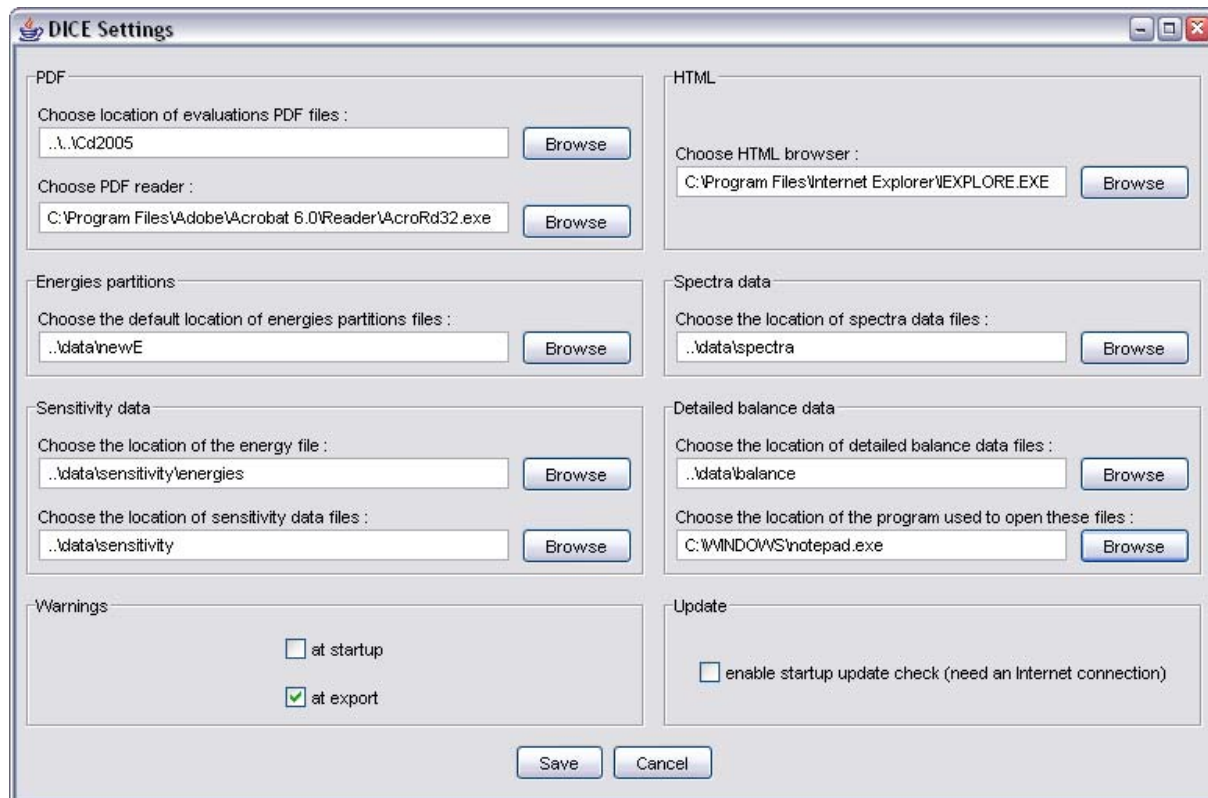
Live update

Changes will continue to be made to the software and to the database after the publication of the DVD. A live update option enables the user who has a web connection to automatically check the updates and download the necessary files from the NEA website. However, as this option replaces the old versions of the files, they should not have a read-only status. Thus, running this option requires the software and the database to be installed on a hard drive with "write" permission.

This option is not activated by default. To run the live-update at each startup or at regular time intervals, see the next paragraph (settings) for details. To run the liveupdate instantaneously, choose the menu option "File/check update...". Each time a new version of the software or the database is available on the NEA web, the program downloads it and replaces the old version. The old version is not deleted but rather renamed. The user can then clean up his installation directory after a successful update by deleting files ended by ". bckdate_and_t i me".

Settings

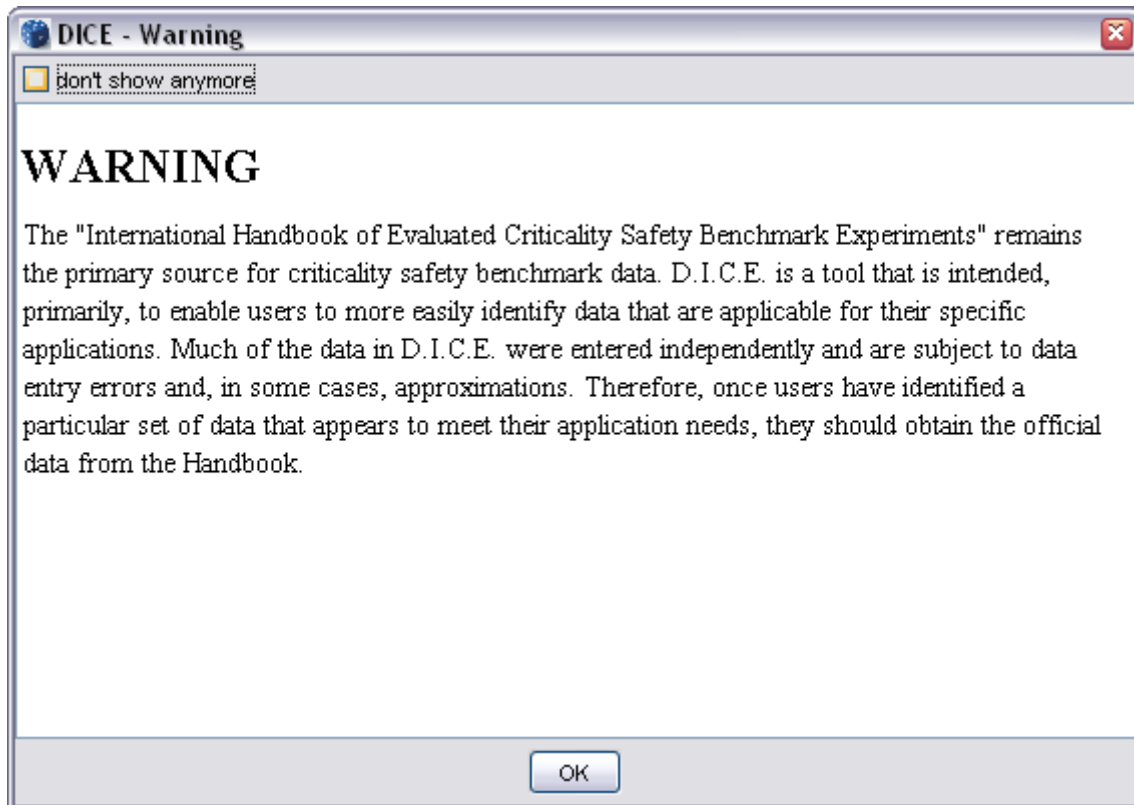
Different default options can be changed using the menu "File/Settings".



This enables the user to specify:

- *The directory where the PDF files of the ICSBEP handbook are located.* The default directory is Cd2005 located at the same level as the di ce. bat file.
- *The path to the PDF reader.* The default viewer is automatically launched on Windows. For other operating systems or if the user wishes to launch a different version of Acrobat, the complete path to the viewer should be specified.
- *The path to the HTML viewer.* As for the PDF reader, the default web browser is automatically launched on Windows. For other operating systems or if the user wishes to launch a different program to view the summary tables, the complete path to the program should be specified.
- *The name of the directory of the spectra data files.* This directory contains sub-directories named after the fuel type (e.g. PU, HEU, LEU...) and the actual file names are of type HST001.001, i.e. a representation of the configuration name.
- *The path to files describing energy schemes.*
- *The name of the directory where the sensitivity files are stored.* This directory contains sub-directories named after the fuel type (e.g. PU, HEU, LEU...) and the actual file names are of type HST001.001, i.e. a representation of the configuration name.
- *The name of the directory where the detailed balance data are stored.*

- *The path to the desired text files editor to be used when opening the detailed balance data files. If nothing is specified, the default Java text editor is used.*
- *The display of the warning pop-up windows (see below) at startup and at results export. Note that both pop-up windows have an option for deactivating their display (checkbox on the top left of the pop-up windows).*



- *The activation of the live update option.*

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